

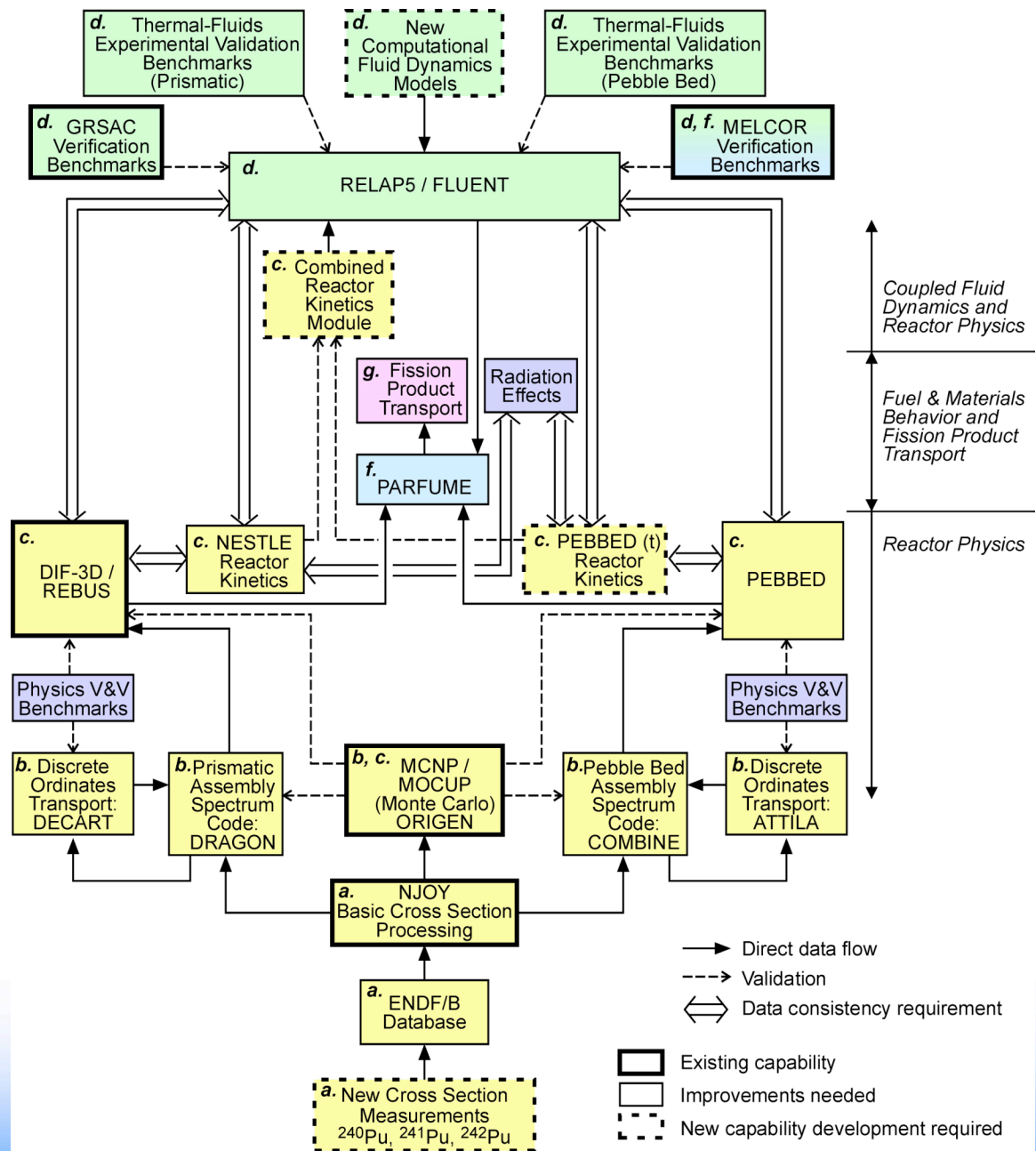
# ***Recent Progress in Computational Methods Development for Pebble Bed Physics and System Modeling***

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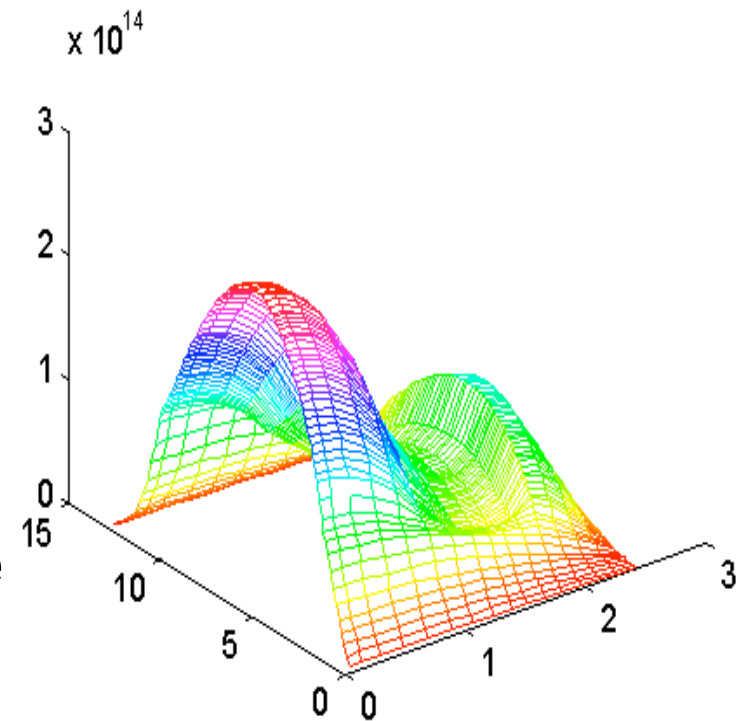
***December 8-9, 2005***

# Proposed VHTR Code Suite



# ***Pebble Bed Reactor Core Physics - Areas of Research and Development***

- PEBBED pebble bed reactor fuel cycle analysis code
- Coupled-Code Development and Cross-section Generation
- Material-Neutronics Interface



# The PBR Fuel Management Problem

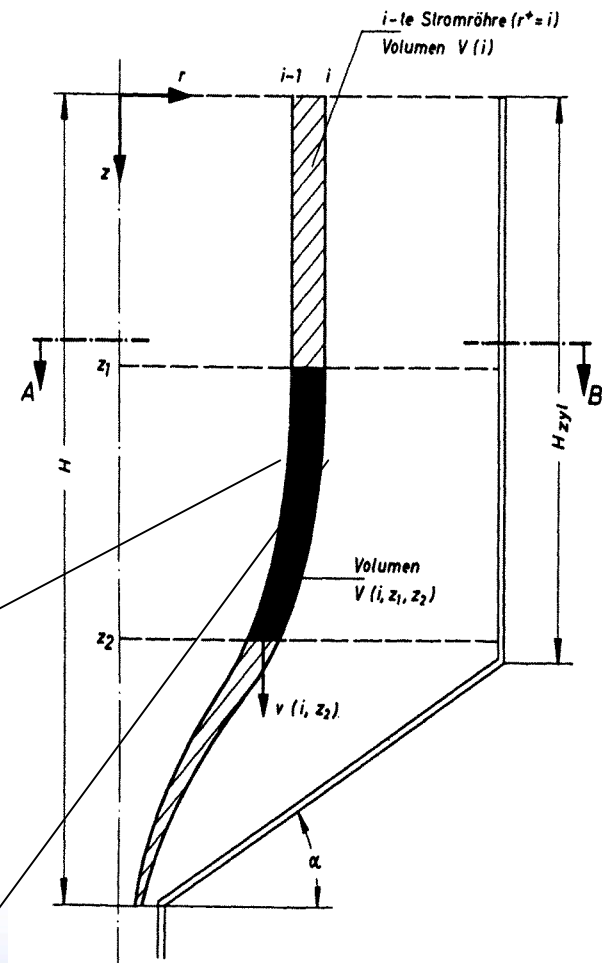
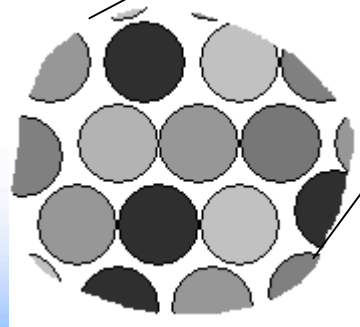
- Fuel flows along streamlines (deterministic component)
- Stochastic component leads to uncertainty
- Partially burned pebbles are recirculated until spent
- Long mean free path, high burnup, harder spectrum



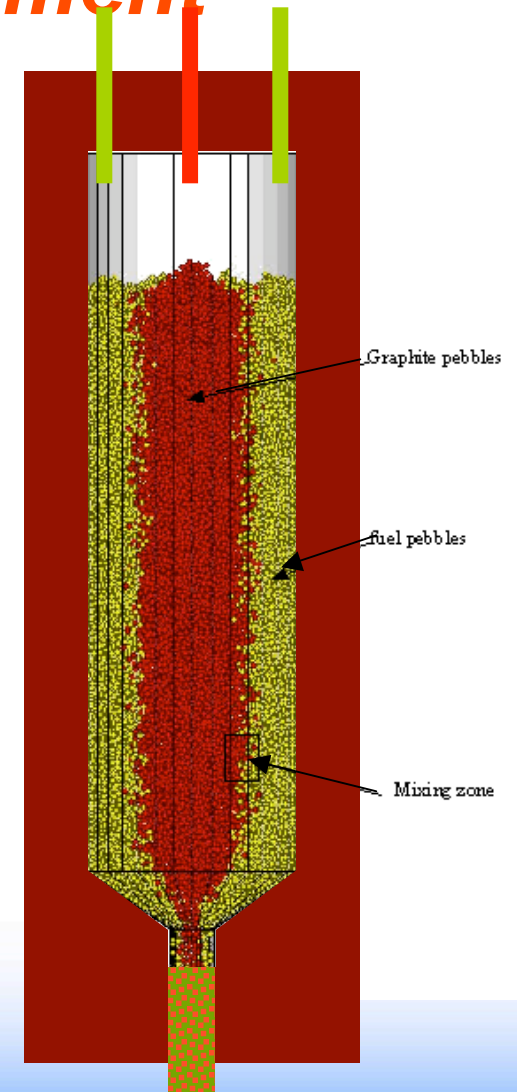
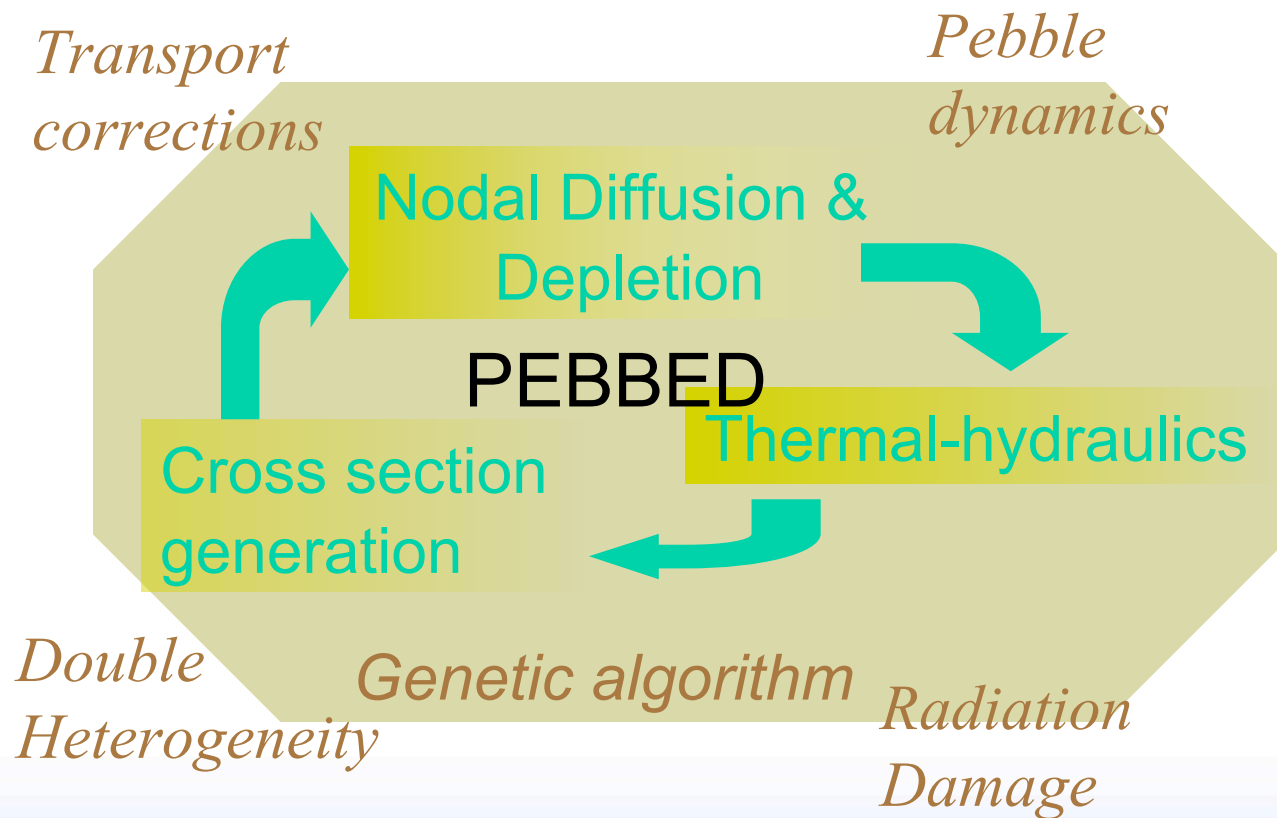
*LWR methods & codes cannot easily be applied*



Idaho National Laboratory

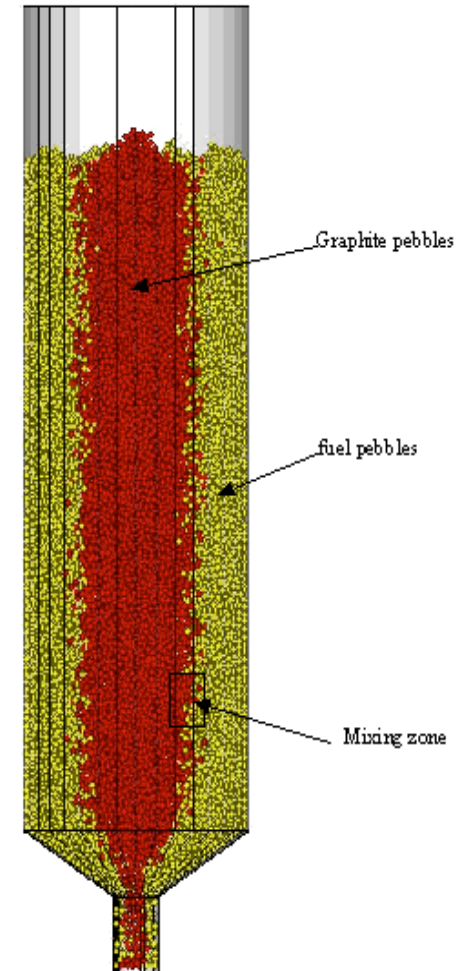


# Pebble Bed Reactor Core Physics - Areas of Research and Development



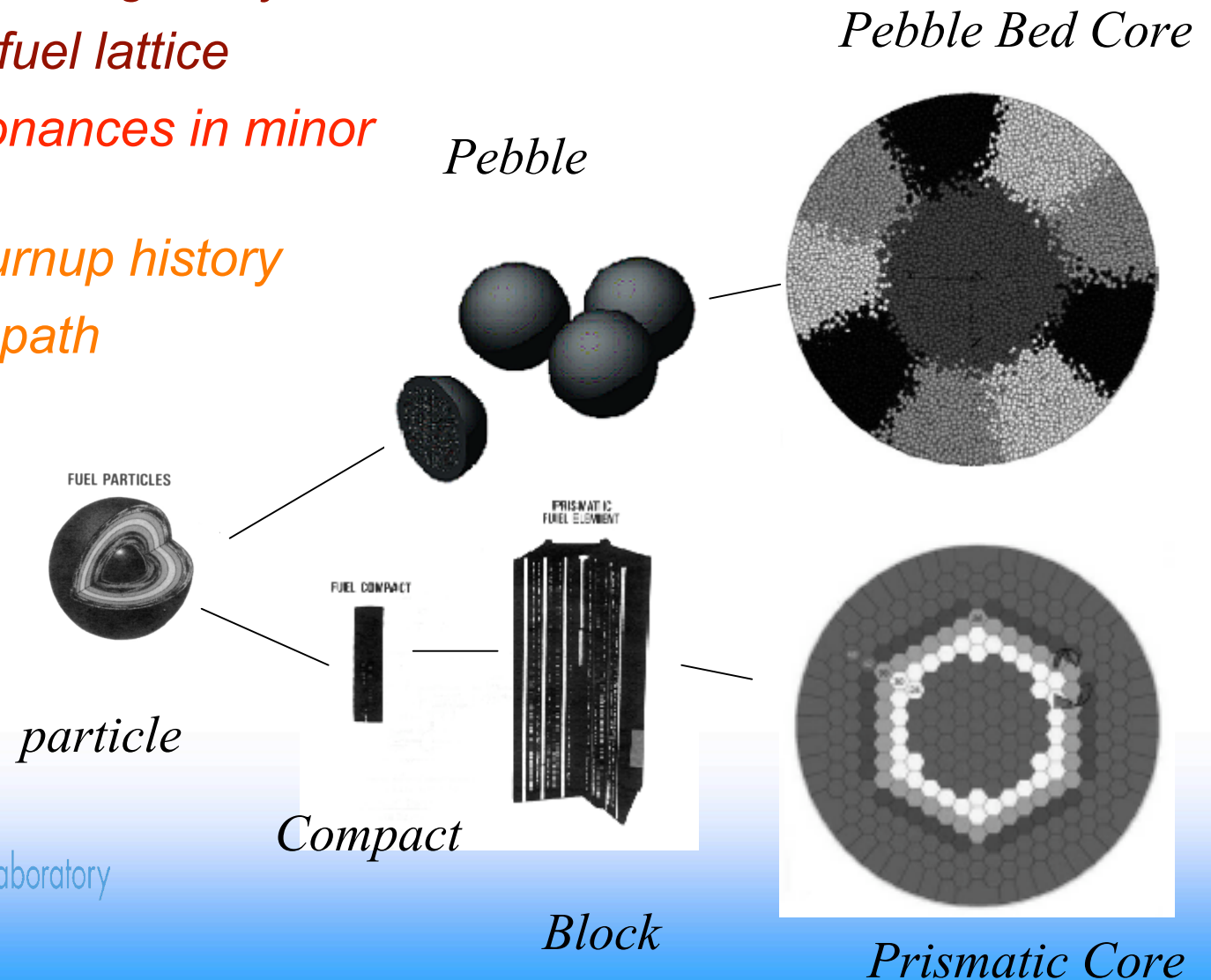
# PEBBED

- Equilibrium Fuel Cycle Analysis of Pebble Bed Reactors
- Finite Difference or Nodal Diffusion
- Low-order thermal-hydraulic modules for normal and accident temperature estimation or THERMIX for accuracy
- Thermal cross section feedback via table lookup or direct calls to a spectrum module
- Arbitrary pebble recirculation schemes can be modeled
- Genetic Algorithm for core geometry and fuel loading optimization



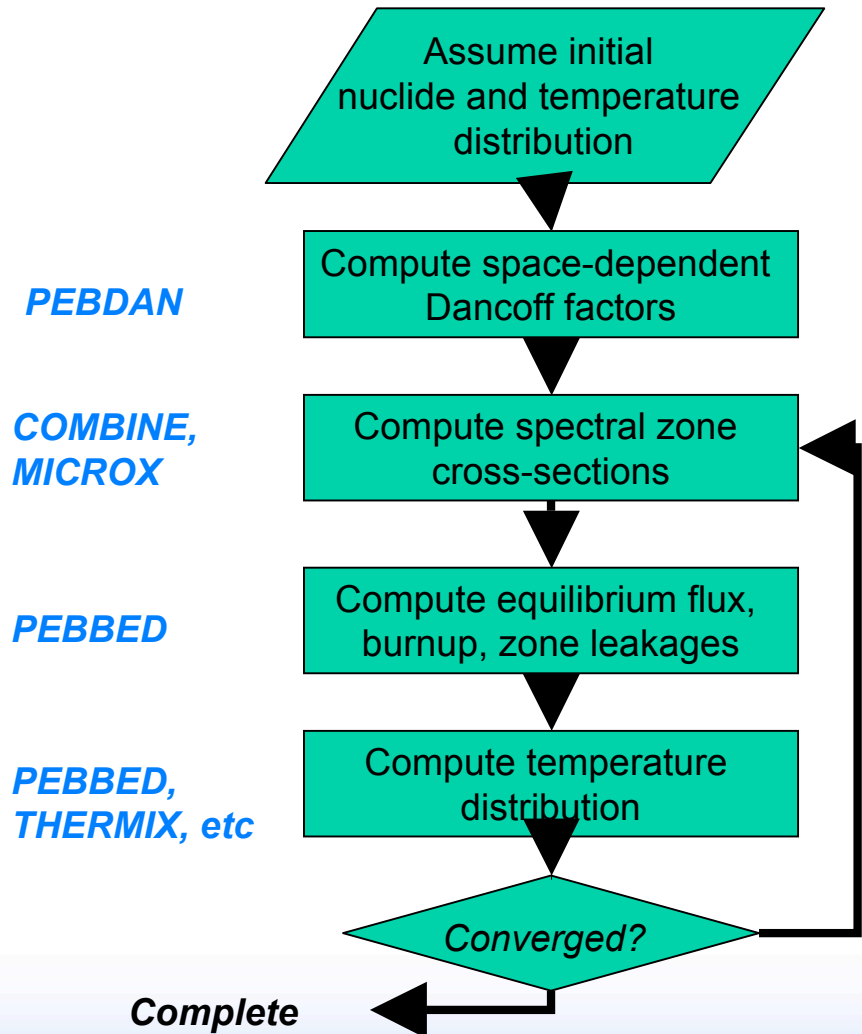
# Cross sections for TRISO-based Fuel

- 2 or 3 levels of heterogeneity
- Randomness in fuel lattice
- Low-energy resonances in minor actinides
- Uncertainty in burnup history
- Long mean free path



# Current Efforts

- Development of Analytical Nodal Diffusion Solver
- Transport Corrections for non-diffusive regions
- Improved Cross-section generation with COMBINE
- Transient Analysis with NEM-THERMIX
- Computational Pebble Flow Dynamics
- Calculation of Displacement Threshold Energy/Neutronic Effects of Radiation Damage and Annealing



*Integrated PBR Fuel Cycle Analysis*



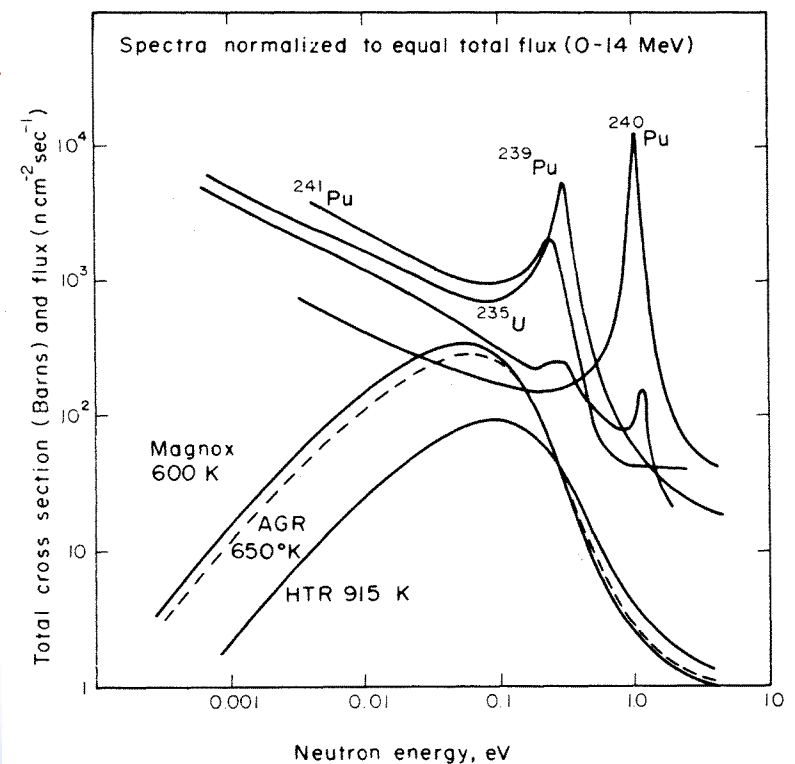
## COMBINE (ENDF/B v6) Developments

- Proper treatment of low energy resonances, energy dependent bucklings, quantum levels in resolved resonances, lumped fission parameters)
- Double-heterogeneity with a new method for computation of spatially-dependent Dancoff factors
- Efficient (on-line) coupling to PEBBED

## Outstanding Issues

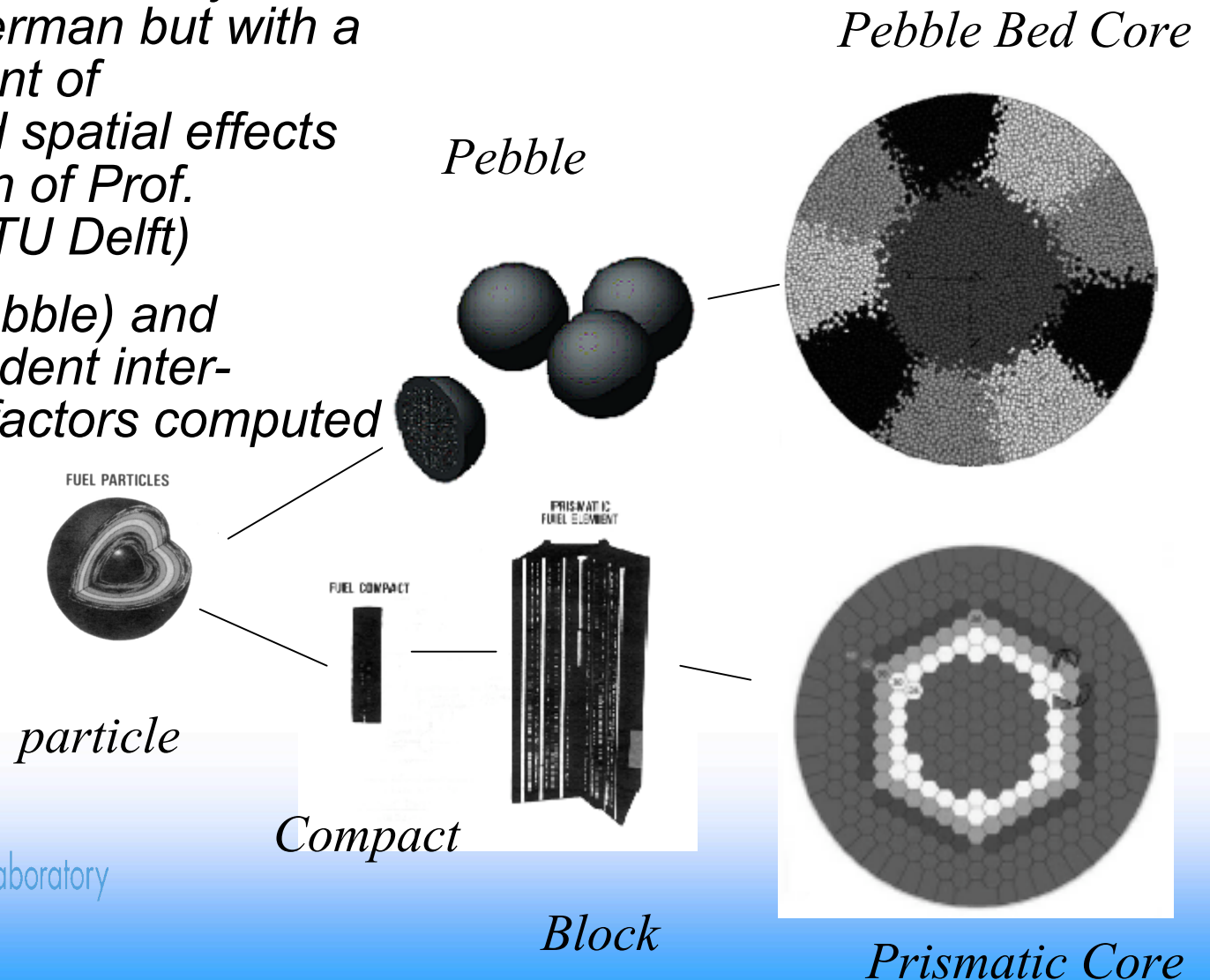
- *Leakage effect between nodes*
- *Proper generation of cross sections in non-multiplying media (reflectors, voids)*

## Integrated Cross-section Generation



# Dancoff Factors for VHTR Fuel

- Builds on previous work by Bende & Kloosterman but with a rigorous treatment of randomness and spatial effects (with cooperation of Prof. Kloosterman of TU Delft)
- Particle (intra-pebble) and geometry-dependent inter-pebble Dancoff factors computed



# *Integrated transport treatment for non-diffusive nodes*

- Response Matrix (transport) approach under investigation (*Georgia Institute of Technology*)
- Preliminary results look promising – an RZ transient diffusion solver with embedded RM is being tested

Reactor Pressure  
Vessel

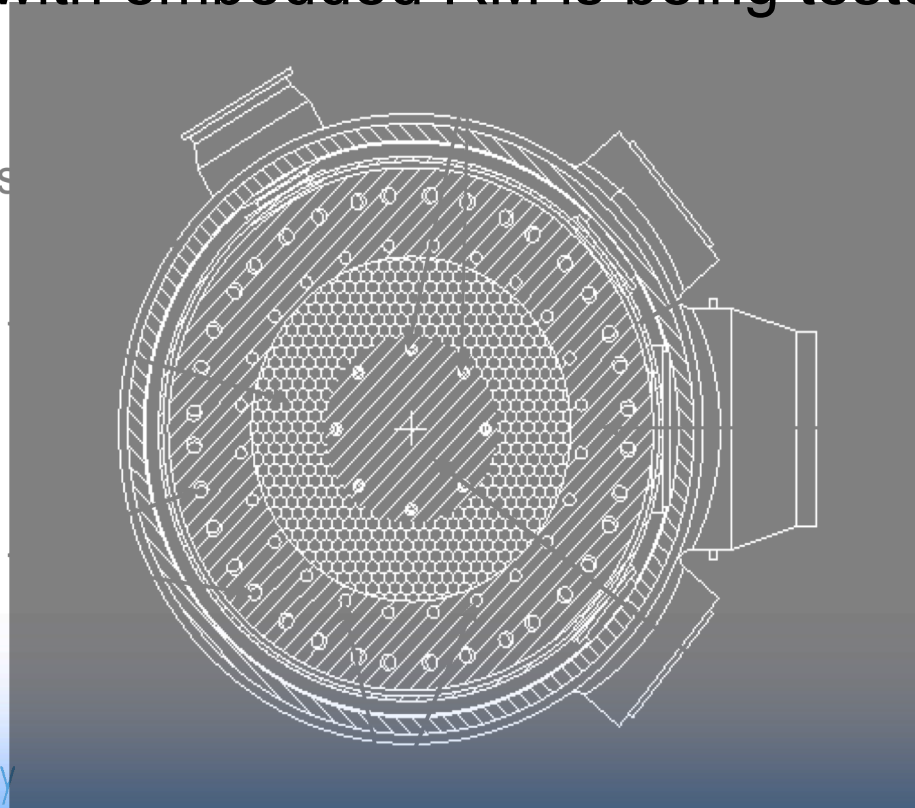
Fuel Core

Gas Riser  
Channels

Pressure Barrel

Guide  
Reflector

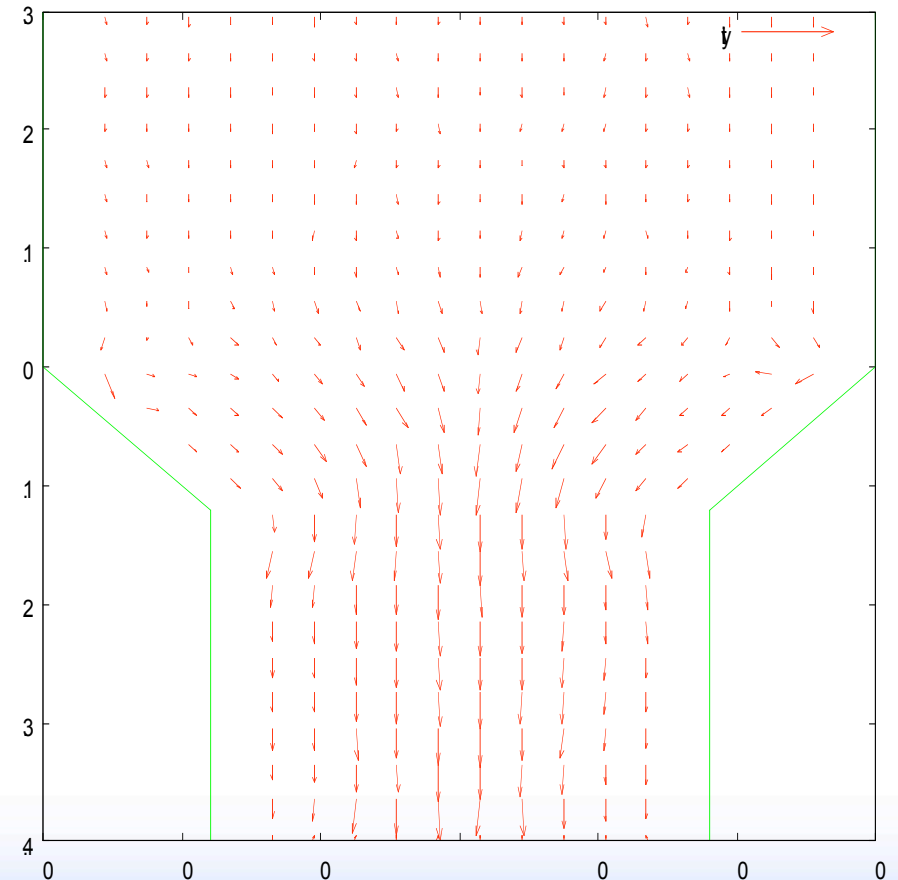
Reflector



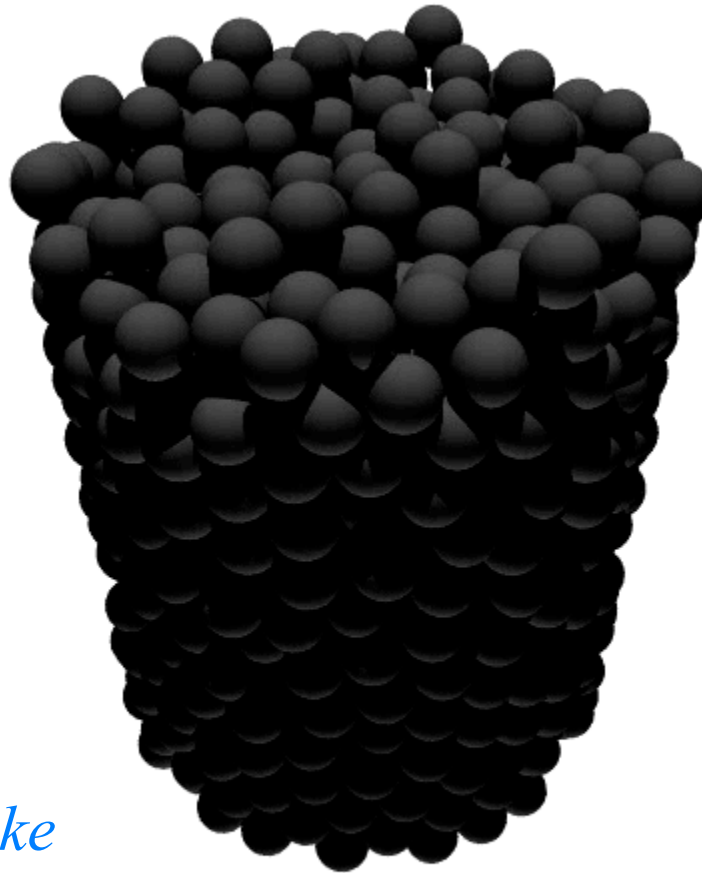
Control Rod Channels

# Pebble Flow Simulation

- *Hard particle dynamics simulation of core loading and settling*
- *Variable packing fraction and pebble flow simulated **but not validated for graphite***
- *Dynamics of pebble flow to drive time-dependent fuel loading analysis capability (Approach to Asymptotic)*
- *Experimental validation and determination of parameters are needed*

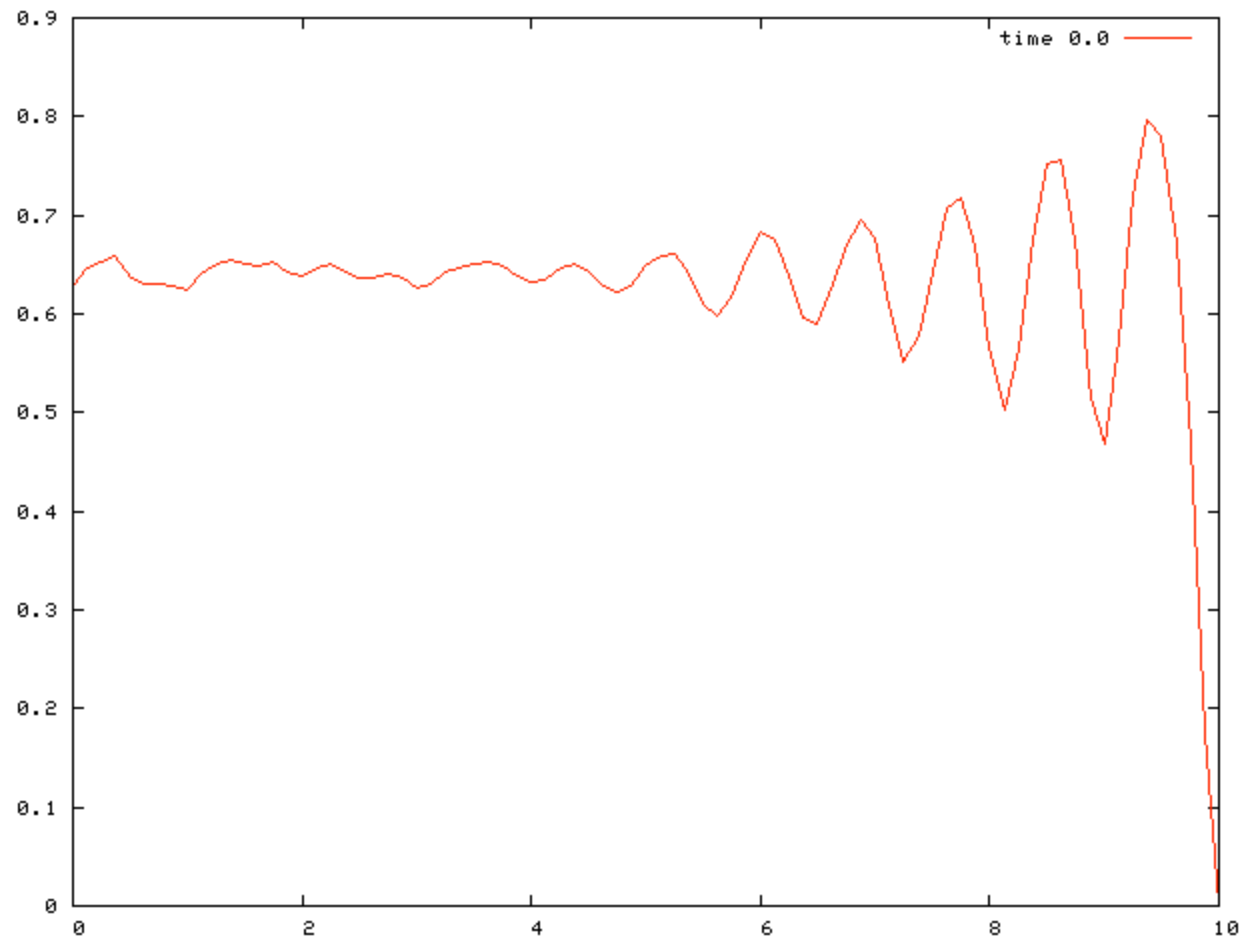


# ***Pebble Settling during a Seismic Event***

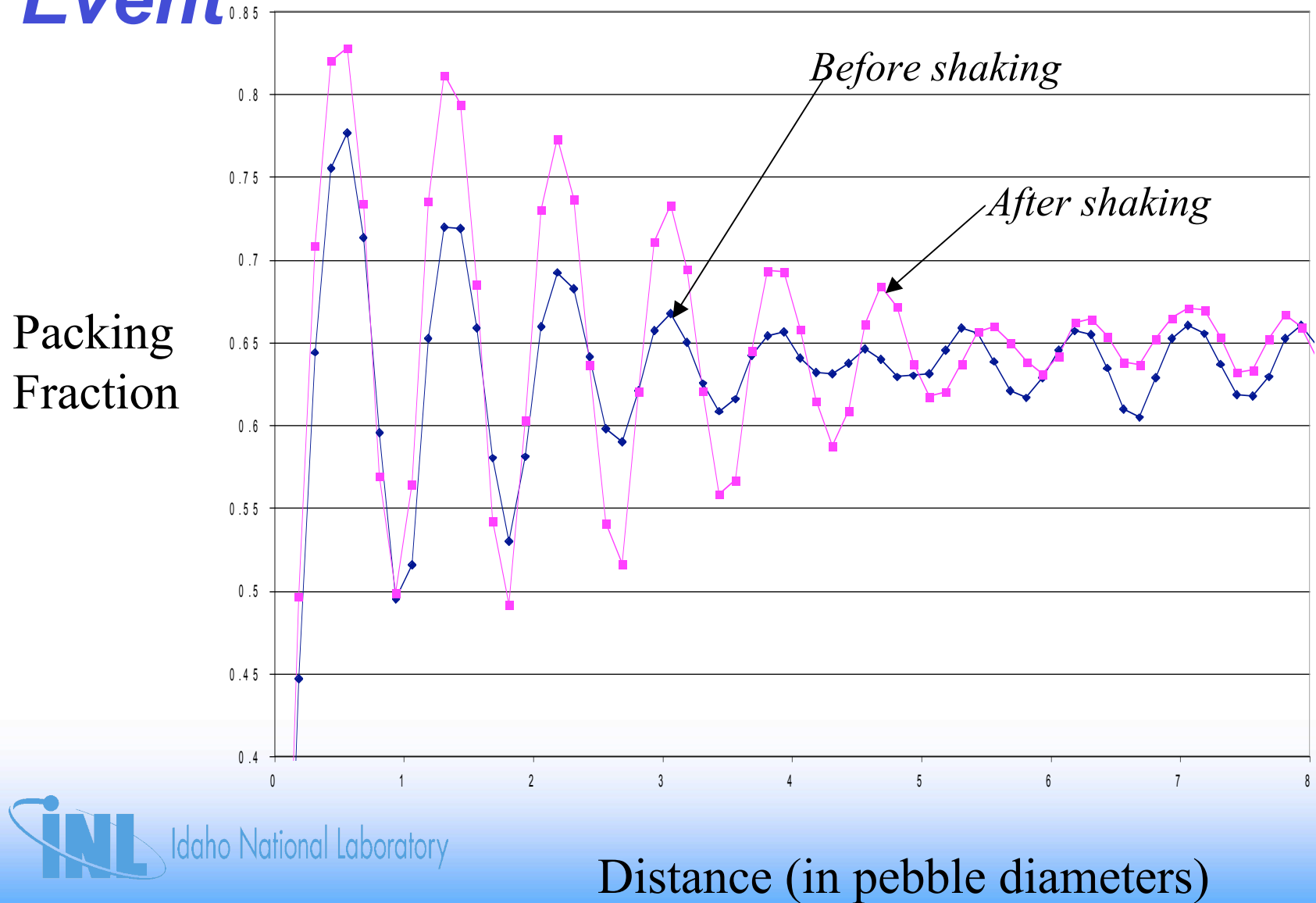


*1000 pebbles in vat  
Mercalli X horizontal quake*

# ***Pebble Settling during a Seismic Event***



# Pebble Settling during a Seismic Event



# *Material-Neutronics Interface – modeling radiation damage and its effects on core dynamics*

- Molecular dynamic simulation of radiation damage in graphite and SiC
- Calculation of scattering cross sections in damaged crystals
- Calculation of threshold energy and displacement kerma cross sections at prototypical temperatures
- Modeling of damage to assess effects of annealing on transient behavior (thermal conductivity recovery – thermal scattering law changes)

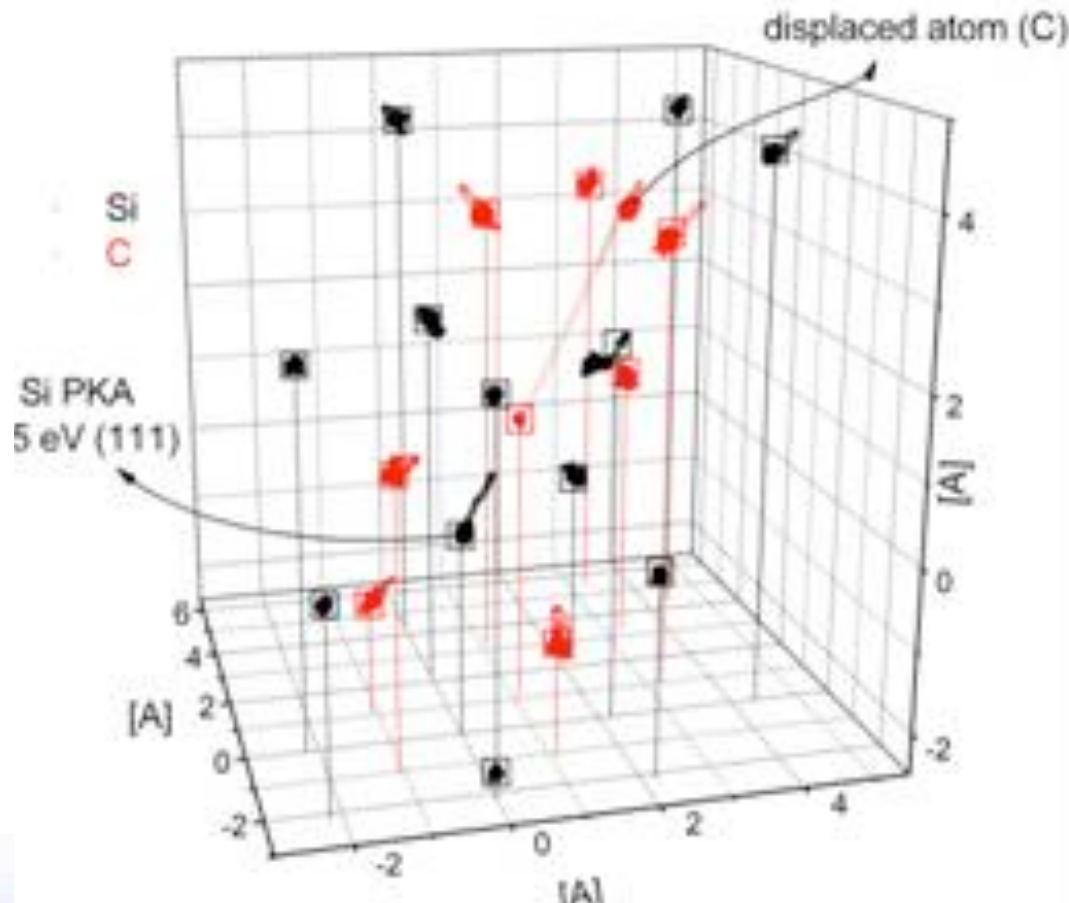


$$E = \frac{I}{2} \sum_{i \neq j} \left\{ f_c(r_{ij}) \left[ f_R(r_{ij}) + b_{ij} f_A(r_{ij}) \right] \right\}$$

$$f_c(r_{ij}) = \begin{cases} 1 & , r_{ij} < R_{ij} \\ \frac{1}{2} \left\{ 1 + \cos \left[ \pi \frac{(r_{ij} - R_{ij})}{(S_{ij} - R_{ij})} \right] \right\} & , R_{ij} < r_{ij} < S_{ij} \\ 0 & , S_{ij} < r_{ij} \end{cases}$$

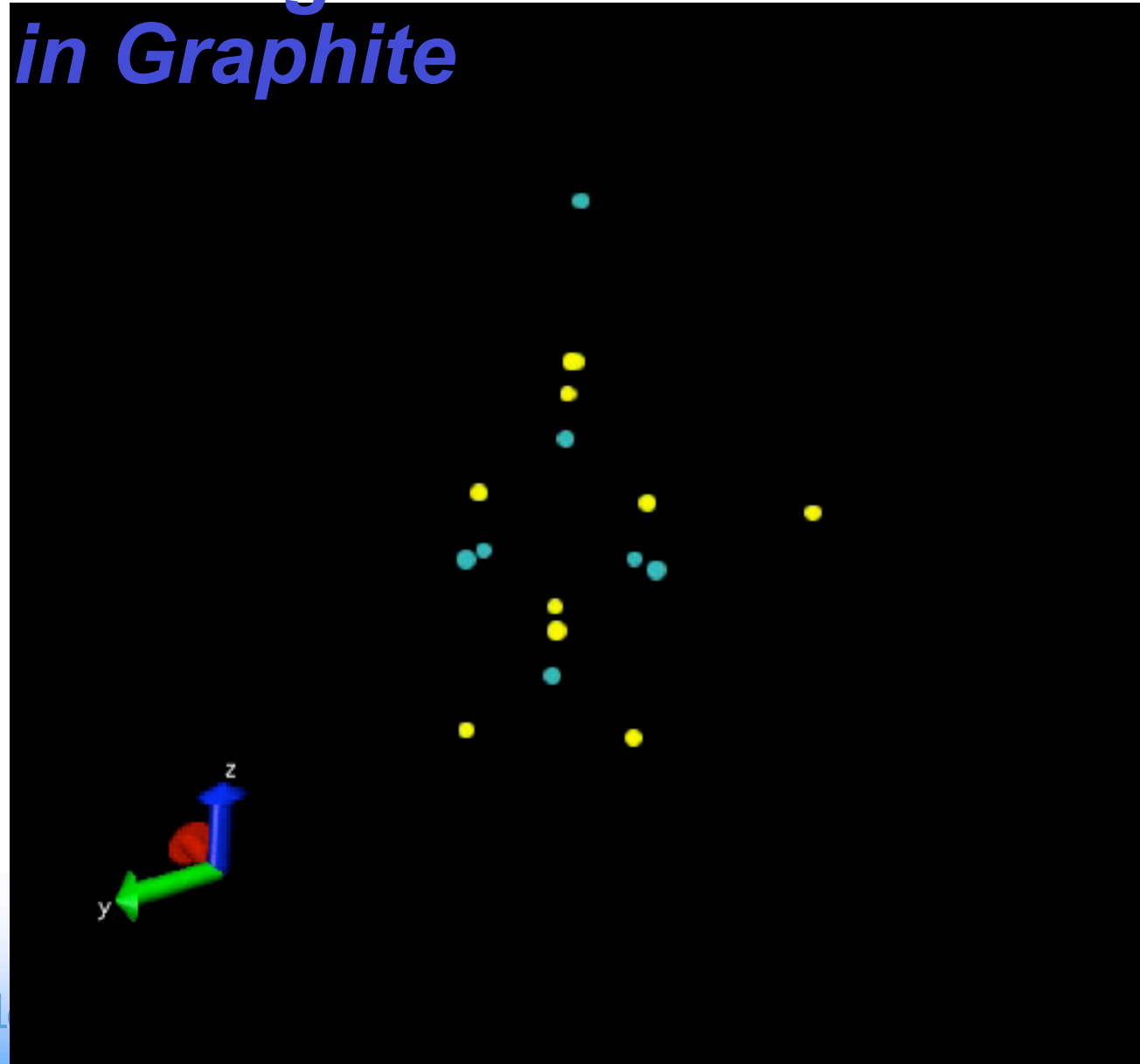


# Dynamic modeling of radiation damage



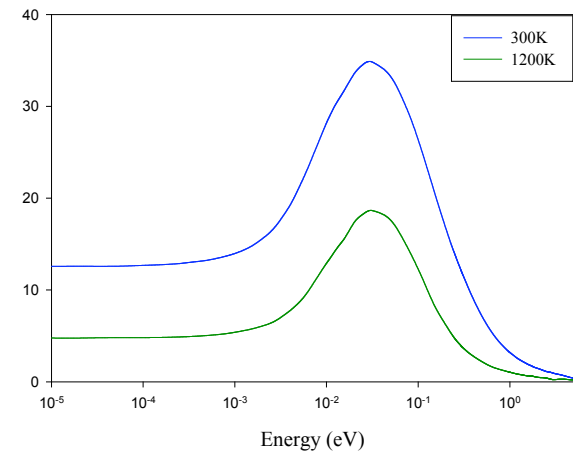
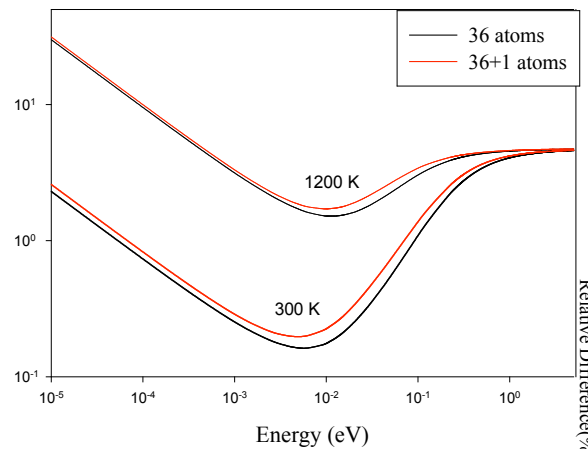
*MDCASK model  
of atomic  
movement in SiC*

# ***Kinetic Modeling of Irradiation Damage in Graphite***

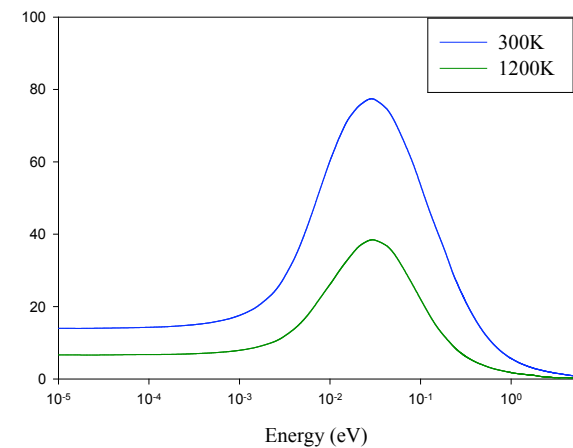
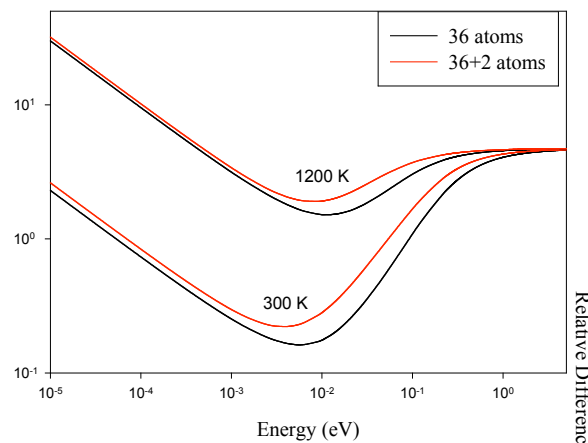


# Effect of radiation damage on scattering cross section

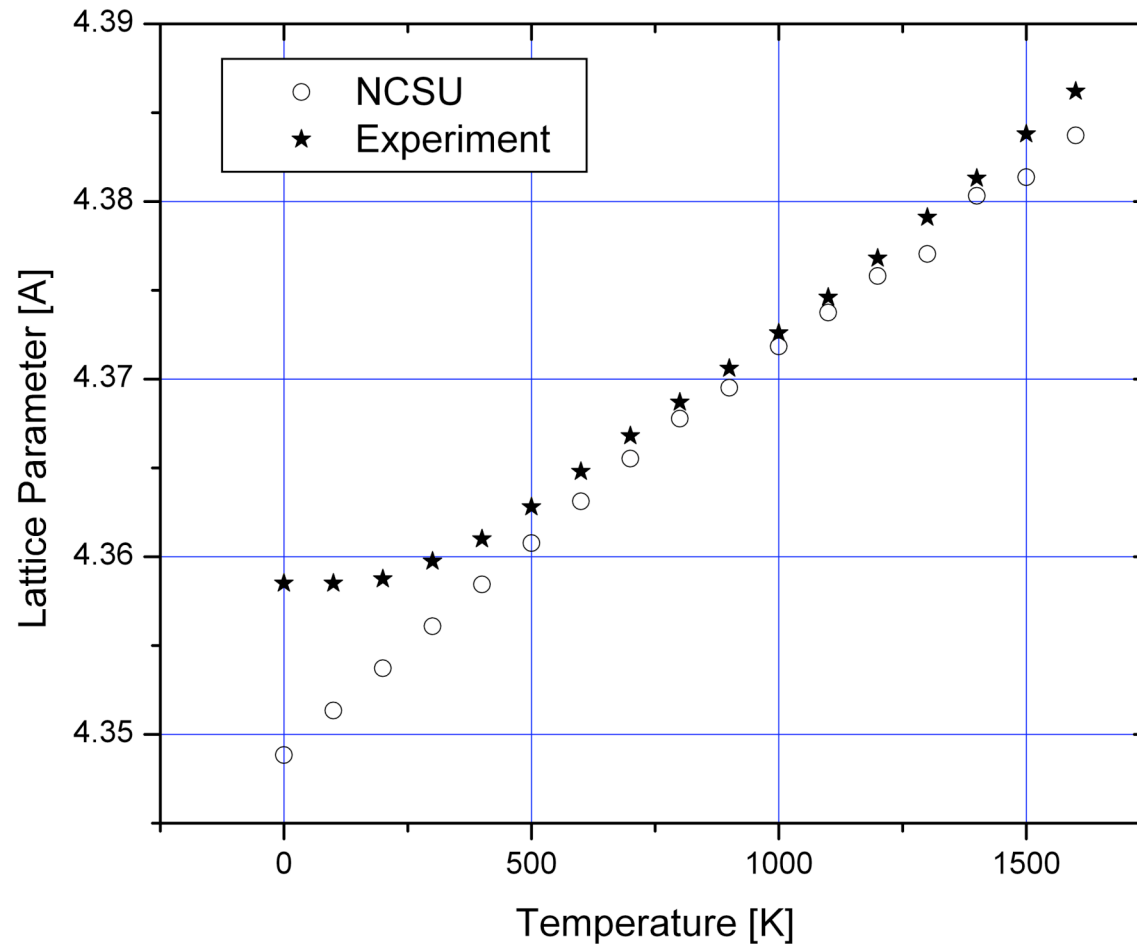
1 interstitial atom



2 interstitial atoms



# *Prediction of lattice parameters*



# Summary

- PEBBED for state-of-the-art PBR analysis and design
  - Advanced optimization
  - Nodal Diffusion
- Coupled-Code Development and Implementation
  - NEM-THERMIX (thermal)
  - COMBINE (cross sections)
- Radiation effects on Core Dynamics
- 2006 – Validation and benchmarking of results

